



**Institute of Fundamental Technological Research  
Polish Academy of Sciences**



# **LOW - ACTIVATION CONCRETE**

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RADCON meeting, 11-14 March, 2019, Smolenice – Slovak Academy of Sciences

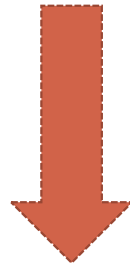
# Outline

- **Goal of the research**
- **Research plan**
- **Expected results**



# Motivation

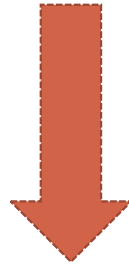
- **Collaboration within RADCON Project**



**multi-author articles, papers, ...**

# Goal of the research

**The lowest possible activated cement based material**



**Low-activation concrete**

# What is already known



Journal of Nuclear Science and Technology



ISSN: 0022-3131 (Print) 1881-1248 (Online) Journal homepage: <https://www.tandfonline.com/loi/tnst20>

## Raw Materials for Low-Activation Concrete Neutron Shields

Masaharu KINNO , Ken-ichi KIMURA & Takashi NAKAMURA

To cite this article: Masaharu KINNO , Ken-ichi KIMURA & Takashi NAKAMURA (2002)

- **the raw materials for low-activation concrete were: low-activation limestone, quartzite, colemanite, alumina-ceramics, white Portland cement and high-alumina cement**
- **concrete surrounding a nuclear reactor**

# What is already known

Fujita Technical Research  
Report No.43 2007



## Low-Activation Reinforced Concrete Design Methodology

-Fundamental Investigation for various Types of Low-Activation Concrete-

Ken-ichi Kimura, Masaharu Kinno, Hirokazu Nishida, Noritcika Katayose,  
Mikihiro Nakata<sup>2)</sup>, Katsumi Hayashi<sup>3)</sup>, Mikio Uematsu<sup>4)</sup> and Akira Hasegawa<sup>1)</sup>

- **3 kinds of aggregates (fused alumina ceramics, silica sand and limestone) and 2 kinds of cements (high alumina cement and white cement) were selected as raw materials for the low-activation concrete**
- **long-term use in a nuclear power plant**

# What is already known

(19)  (11)  EP 3 266 754 A1

(12) EUROPEAN PATENT APPLICATION

(43) Date of publication: 10.01.2018 Bulletin 2018/02 (51) Int Cl.: C04B 28/02 (2006.01) C04B 28/06 (2006.01) G21F 1/04 (2006.01)

(21) Application number: 16178315.4 (22) Date of filing: 07.07.2016

(84) Designated Contracting States: AL AT BE BG CH CY CZ DE DK EE ES FI FR GB GR HR HU IE IS IT LI LT LU LV MC MK MT NL NO PL PT RO RS SE SI SK SM TR Designated Extension States: BA ME Designated Validation States: MA MD	(72) Inventors: • STICHELBAUT, Frédéric 1348 Louvain-La-Neuve (BE) • PIÉARD, Julie 1000 Brussels (BE) • KUPERS, Laura 1000 Brussels (BE)
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(54) COMPOSITIONS OF LOW ACTIVATION CONCRETE AND USE THEREOF

- high-purity limestone aggregate and white cement
- high-purity limestone aggregate and aluminous cement
- the present invention relates to the use of low-activation concrete in radiation protection structures, preferably for particle accelerators

# What is already known

Application of low-activation concrete



Neutron-shielding (biological shielding)

## What about concrete properties?

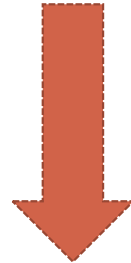
**The concrete with high alumina cement:**

- **high drying shrinkage**
- **long term durability (low w/c ratio)**
- **heat generation**



# Goal of the research

**The lowest possible activated cement based material**



**Low-activation concrete**

- **Cask storing spent fuel (low-activation concrete)**
- **Long term durability of concrete (liquid and gaseous media)**

# Cask storing spent fuel



**the Yankee NPP, Connecticut**

HI-STORM is engineered for maximum shielding, ... twenty-six inches of concrete (0.66 m)



<https://holtecinternational.com>

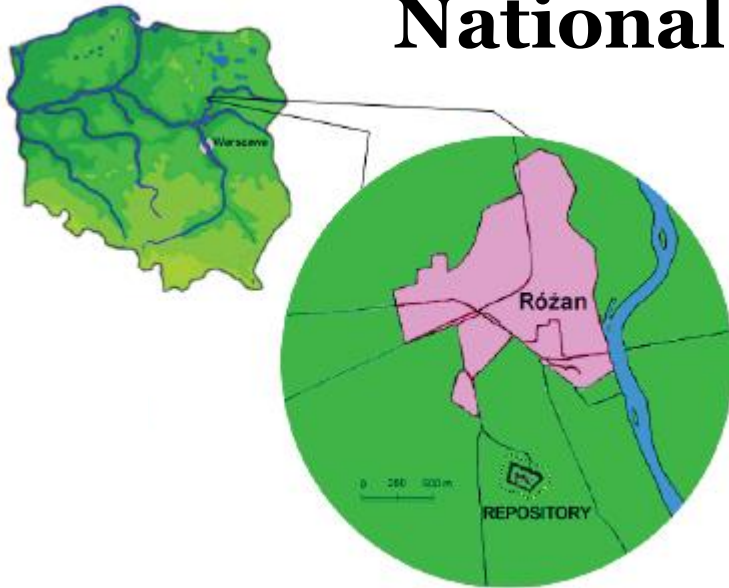
**the Zion NPP, Illinois**

<http://www.zionsolutionscompany.com>



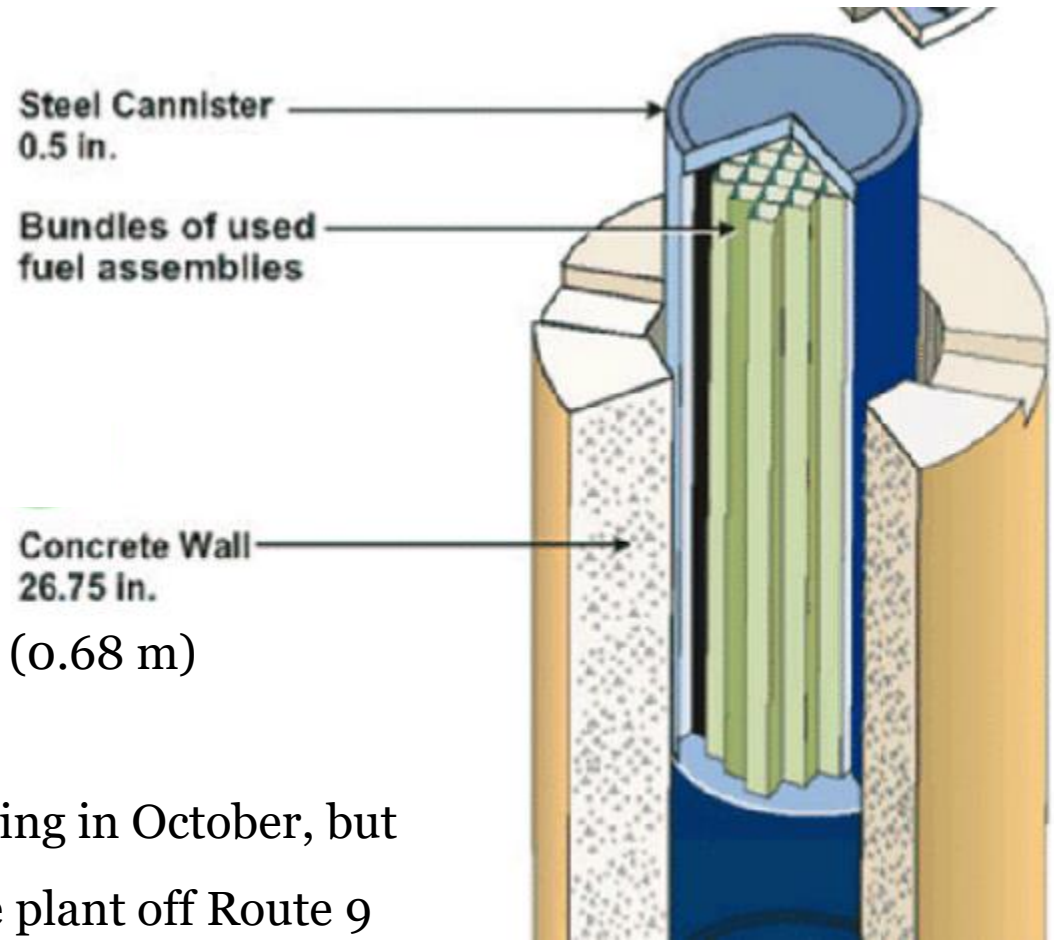
*Low-activation concrete*

# National Radioactive Waste Deposit Poland



*Low-activation concrete*

# the Oyster Creek NPP, New Jersey



The Oyster Creek NPP may be closing in October, but the spent nuclear fuel stored at the plant off Route 9 here for nearly 50 years isn't going anywhere.

Feb 6, 2018

<https://patch.com/new-jersey/lacey/oyster-creeks-spent-nuclear-fuel-casks-arent-going-anywhere>

# Goal of the research

**The lowest possible activated and durable cement based material for cask storing spent fuel**

**The design of the low-activation or even non-activation concrete**

# What we are looking for?

- **The trace elements predominantly responsible for the activation of concrete materials**
- **Eu and Co are dominant materials for activation in the cement and concrete**

The three radioisotopes  $^{60}\text{Co}$ ,  $^{152}\text{Eu}$  and  $^{154}\text{Eu}$  alone occupy 99–100% of the total residual radioactivity in ordinary concrete at the time of decommissioning

**Radiochemical analyses: Co, Eu, Cs, Fe and Sc**

# Research plan

- **Selection of aggregates, cements and additives (fly ash, slag, ...) as raw materials to design and perform the low-activation concrete**
- **Evaluation of materials by radiochemical analyses (NAA, PGAA ???) which assessed the quantities of dominant trace elements for the activation under specific conditions**
- **Design and perform of special concrete for cask storing spent fuel**
- **Properties of fresh mix (workability, heat generation, ...)**
- **Fundamental mechanical properties**
- **Durability test (gas and water permeability, ...)**

# Expected results

- 1. Activation analysis of cements, aggregates and additives (fly ash, slag, ...)**
- 2. The design of 4 or 6 of concretes (mortars)**
  - Low-activation concrete**
  - Durable concrete (liquid and gaseous media)**
- 3. Activation analysis of performed concretes**
- 4. Results of the long term durability of concretes**
- 5. Database for Monte Carlo simulations ?**



# Any other ideas?

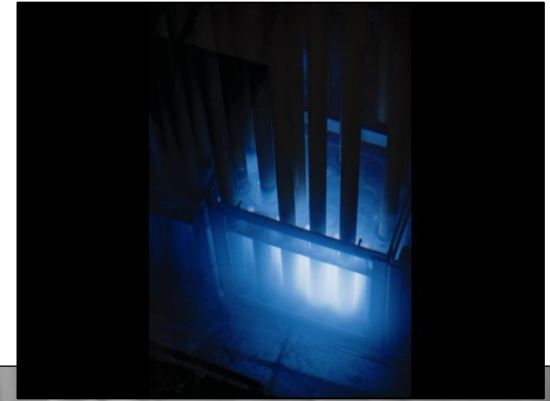
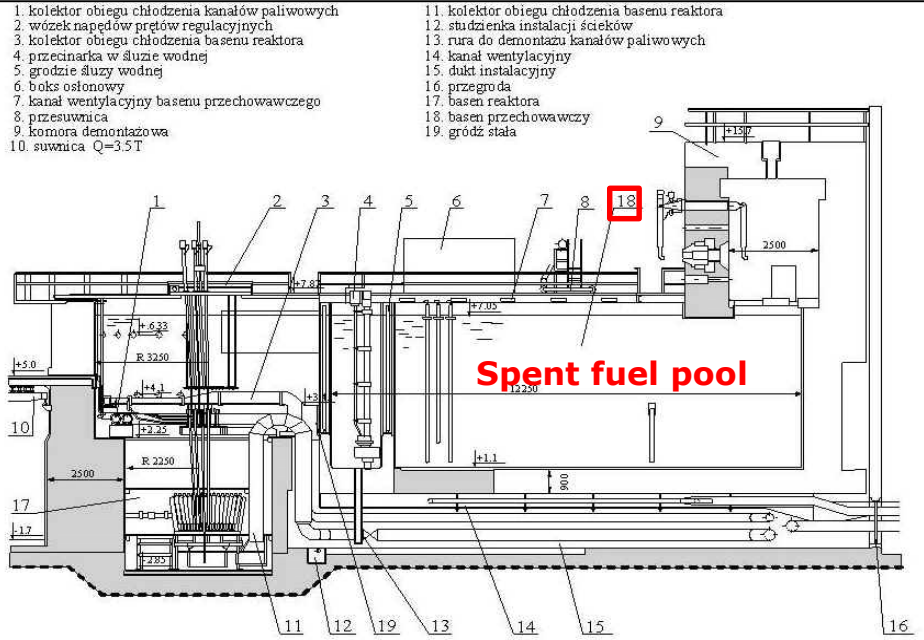
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# Activation analysis of cement and supplementary cementitious materials

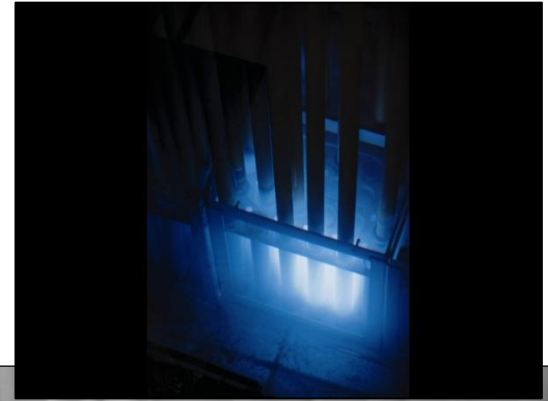
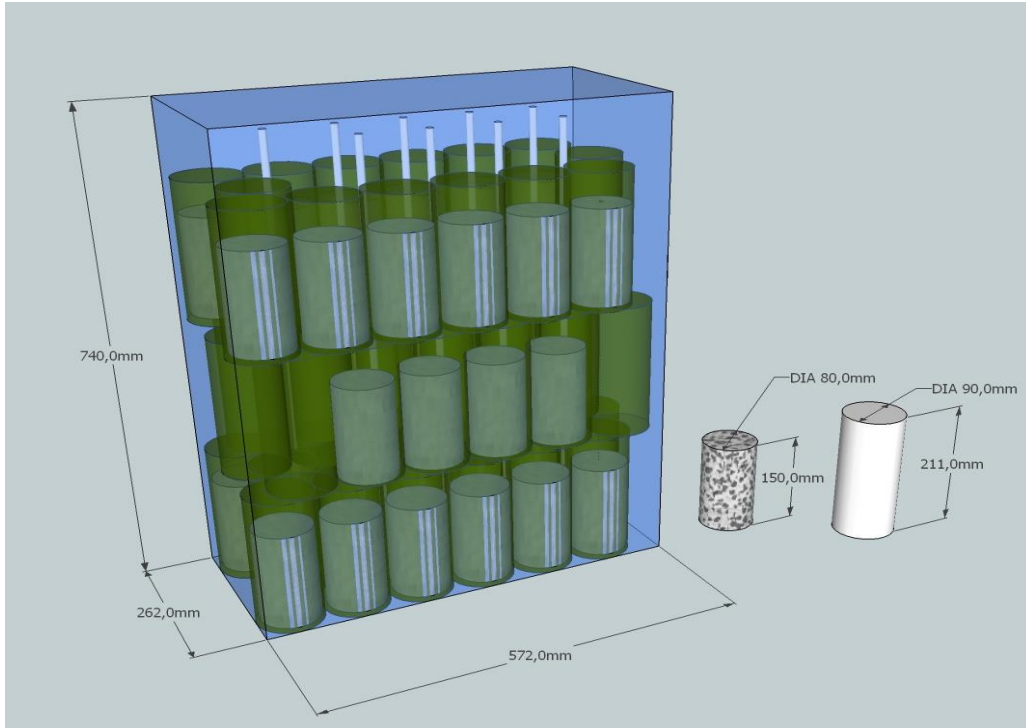
Material	Isotope	$T_{1/2}$	mass [mg]	kBq/g	$10 \times T_{1/2}$ [kBq/kg]
Cem I 42.5R	Sc-46	83.785 d	246	4.9	4.8
	Sc-47	3.349 d		39.2	38.2
	Fe-59	44.495 d		3.9	3.8
	In-114m	1.198 m		0.8	0.8
Ground granulated blast furnace slag	Sc-46	83.785 d	241	15.2	14.89546
	Sc-47	3.349 d		30.3	29.64591
Fly ash	Sc-46	83.785 d	178	27.0	26.41156
	Fe-59	44.495 d		11.7	11.4513
	Co-60	5.274 y		1.8	1.827315

5 min in reactor MARIA, neutron flux  $2 \times 10^{14} \text{ cm}^{-2} \text{ s}^{-1}$

# Irradiation of concrete specimens – separator with spent fuel

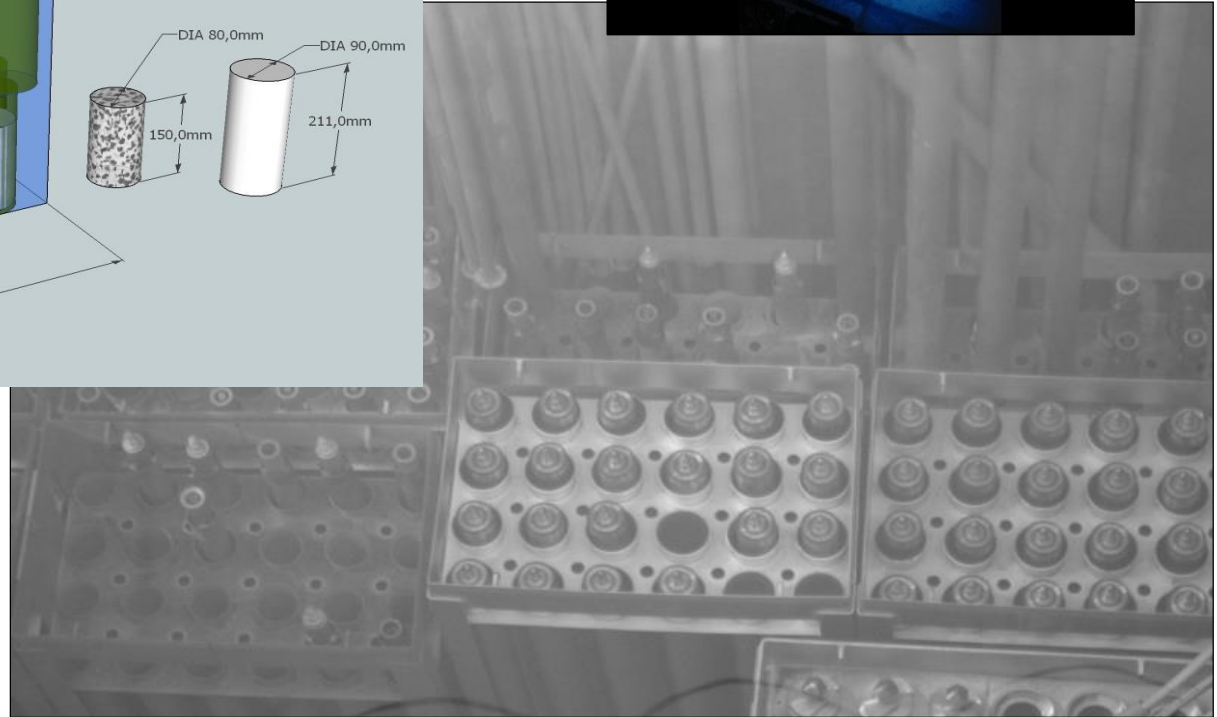


# Irradiation of concrete specimens – separator with spent fuel



**a central sealed container placed in a separator surrounded by 16 fuel elements**

**10 months  $>10^7$  Gy**



# Thank you for your attention

## **Acknowledgements**

The financial support by Polish National Centre for Research and Development (Project V4-506 Korea/2/2018) is gratefully acknowledged.